

## Motivations

- ▶ Nuclear matter emits neutrons, detection is used to identify the type of matter.
- ▶ Extracted data are composed of a timelist file, which is a listing of detection times
- ▶ Given a timelist file, how can we estimate the fissile matter characteristics?

## Neutronic fluctuations

Neutron fluctuations are due to branching processes

- ▶ neutron reactions (fission, capture)
- ▶ source (spontaneous fission)
- ▶ detection

Main random process is the fission branching process.

We analyse the timelist file as follows

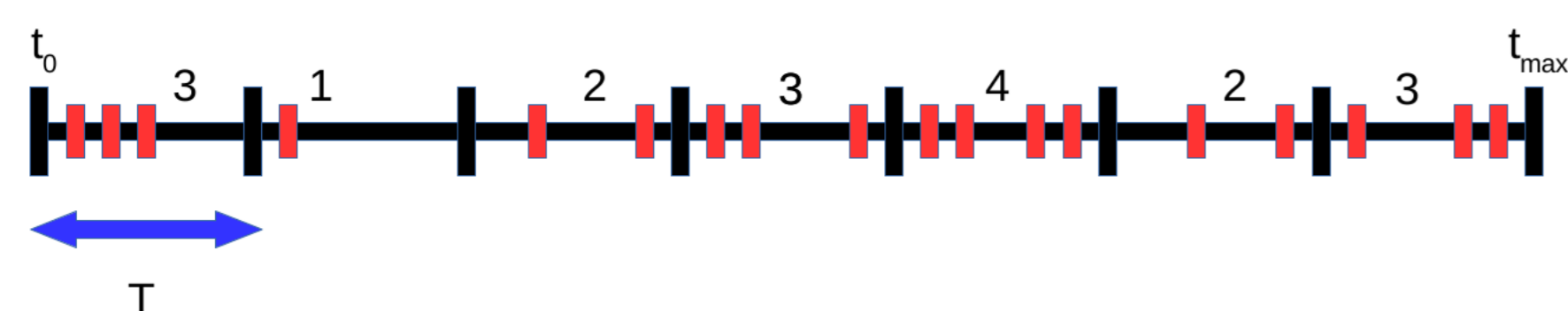


FIGURE – Sequential binning

It produces a count vector from which we compute empirical moments.

## Point model and forward problem

- ▶ A stochastic point model was introduced by R.Feynman during the Manhattan project.
- ▶ More precisely : all the neutrons move at same speed in an infinite homogeneous, isotropic medium.  
A neutron ends its life by capture (with or without detection) or induced fission.  
Neutrons are produced by induced fission and by source which can be a Poisson or compound Poisson process (spontaneous fission).

We consider the following outputs

- ▶ Average count number

$$\bar{C}(T) = \frac{\varepsilon_F S}{-\rho \bar{\nu}} T$$

- ▶ Feynman moment of order 2

$$Y_2(T) = \frac{\varepsilon_F D_2}{\rho^2} \left( 1 - x \rho \frac{\bar{\nu}_S D_{2S}}{\bar{\nu} D_2} \right) \left( 1 - \frac{1 - e^{-\alpha T}}{\alpha T} \right)$$

- ▶ Feynman moment of order 3

$$Y_3(T) = 3 \left( \frac{\varepsilon_F D_2}{-\rho^2} \right)^2 \left( 1 - x \rho \frac{\bar{\nu}_S D_{2S}}{\bar{\nu} D_2} \right) \left( 1 + e^{-\alpha T} - 2 \frac{1 - e^{-\alpha T}}{\alpha T} \right) - \frac{\varepsilon_F D_3}{\rho^3} \left( 1 - x \rho \frac{\bar{\nu}_S^2 D_{3S}}{\bar{\nu}^2 D_3} \right) \left( 1 - \frac{3 - 4e^{-\alpha T} + 2e^{-2\alpha T}}{\alpha T} \right)$$

## Check of the point model with Monte Carlo codes

Test problem : Fissile solution (UHE+water)

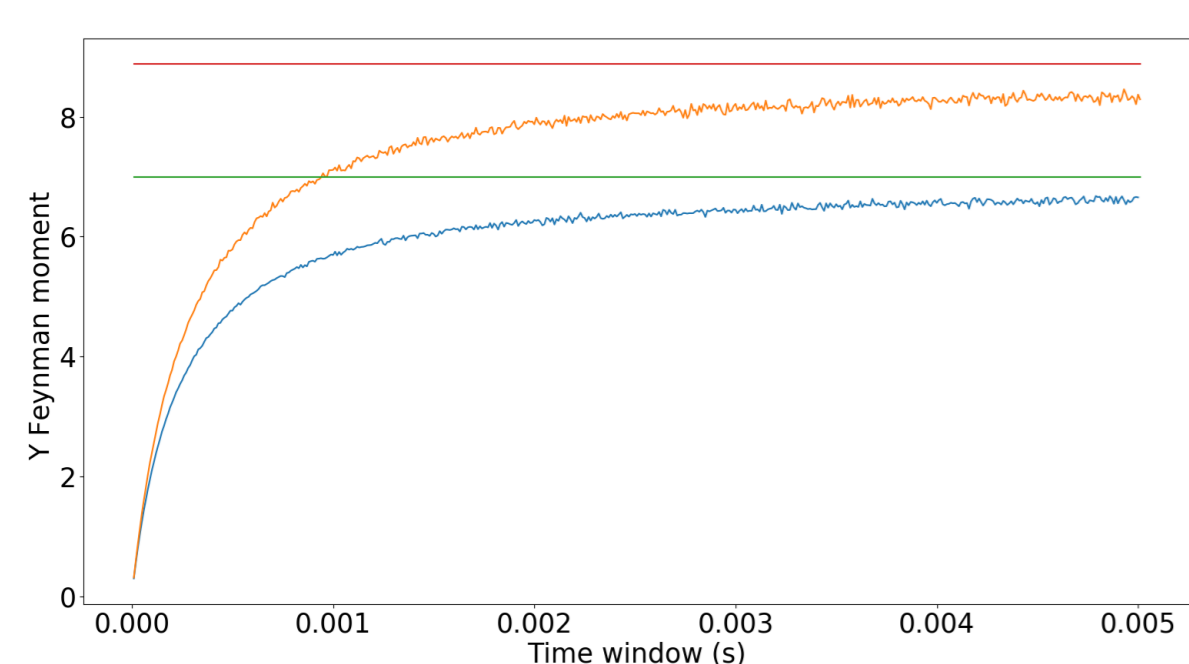


FIGURE – Feynman moment of order 2 for the fissile solution (UHE+water)

- ▶ Tripoli-4 Freya model detailed fission
- ▶ MCNP Terrell type distribution (gaussian)

Test problem :

- ▶  $Cf^{252}$  : Spontaneous fission source, Poisson compound

- ▶ Feynman moment of order 2 in function of the time window (Tripoli-4)
- ▶ Feynman moment of order 2 in function of the time window (MCNP)
- ▶ Asymptotic value of the Feynman moment of order 2

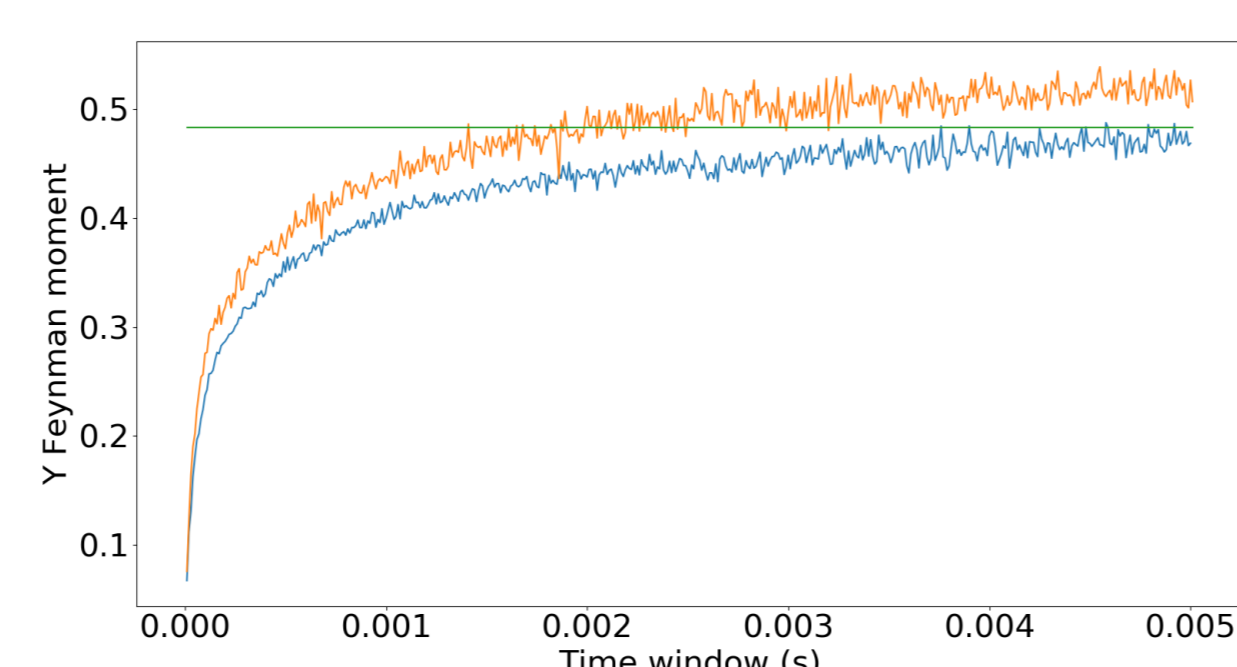


FIGURE – Feynman moment of order 2 for the  $Cf^{252}$

The point model is valid, it can be used for the inverse problem.

## Physical parameters

### Parameters of the model

- ▶  $\varepsilon_F = \frac{\text{count number}}{\text{induced fission number}}$
- ▶  $S$  intensity of the source
- ▶  $\rho$  reactivity of the system
- ▶  $x$  proportion of spontaneous fission source neutrons

### Nuclear Datas

- ▶  $\bar{\nu}$  average number of neutron emitted by a fission
- ▶  $D_i = \frac{\nu(\nu-1)\dots(\nu-i)}{\bar{\nu}^i}$ ,  $i \in \mathbb{N}^*$  Diven factor (idem for  $D_{i,S}$ ,  $i \in \mathbb{N}^*$ )

### Measurements

- ▶ Average count number and Feynman moment

$$\bar{C}(T) = M_1$$

$$Y_2(T) = \frac{2M_2}{M_1} - M_1$$

$$Y_3(T) = \frac{6M_3}{M_1} - 6M_2 + 2M_1^2$$

where  $M_i$ ,  $i \in \mathbb{N}$ , moment of order  $i$  of neutrons detected in  $[0, T]$ .

## The inverse problem, point estimation

We are looking for the  $\mathbf{u}$  which minimizes the mean square error function

$$\|\mathbf{y}_{obs} - \mathbf{f}(\mathbf{u})\|_{\Gamma}^2$$

where  $\mathbf{y}_{obs} = \begin{pmatrix} \bar{C}(T) \\ Y_2(T) \\ Y_3(T) \end{pmatrix}$  are the measures,  $\mathbf{u} = \begin{pmatrix} \varepsilon_F \\ \rho \\ S \\ x \end{pmatrix}$  the parameters, and  $\Gamma$  is

the covariance matrix of the empirical Feynman moments.

We obtain the best estimation  $\hat{\mathbf{u}}$  using optimization methods like simulated annealing.

|                  | Parameters      |           |         | Feynman moments |        |          |
|------------------|-----------------|-----------|---------|-----------------|--------|----------|
|                  | $\varepsilon_F$ | $S$       | $\rho$  | $\bar{C}$       | $Y_2$  | $Y_3$    |
| Given values     | 0.481           | 1000      | -0.258  | 769.75272       | 6.9939 | 117.2060 |
| Estimated values | 0.4614          | 1020.4884 | -0.2526 | 769.7501        | 6.9746 | 117.2123 |

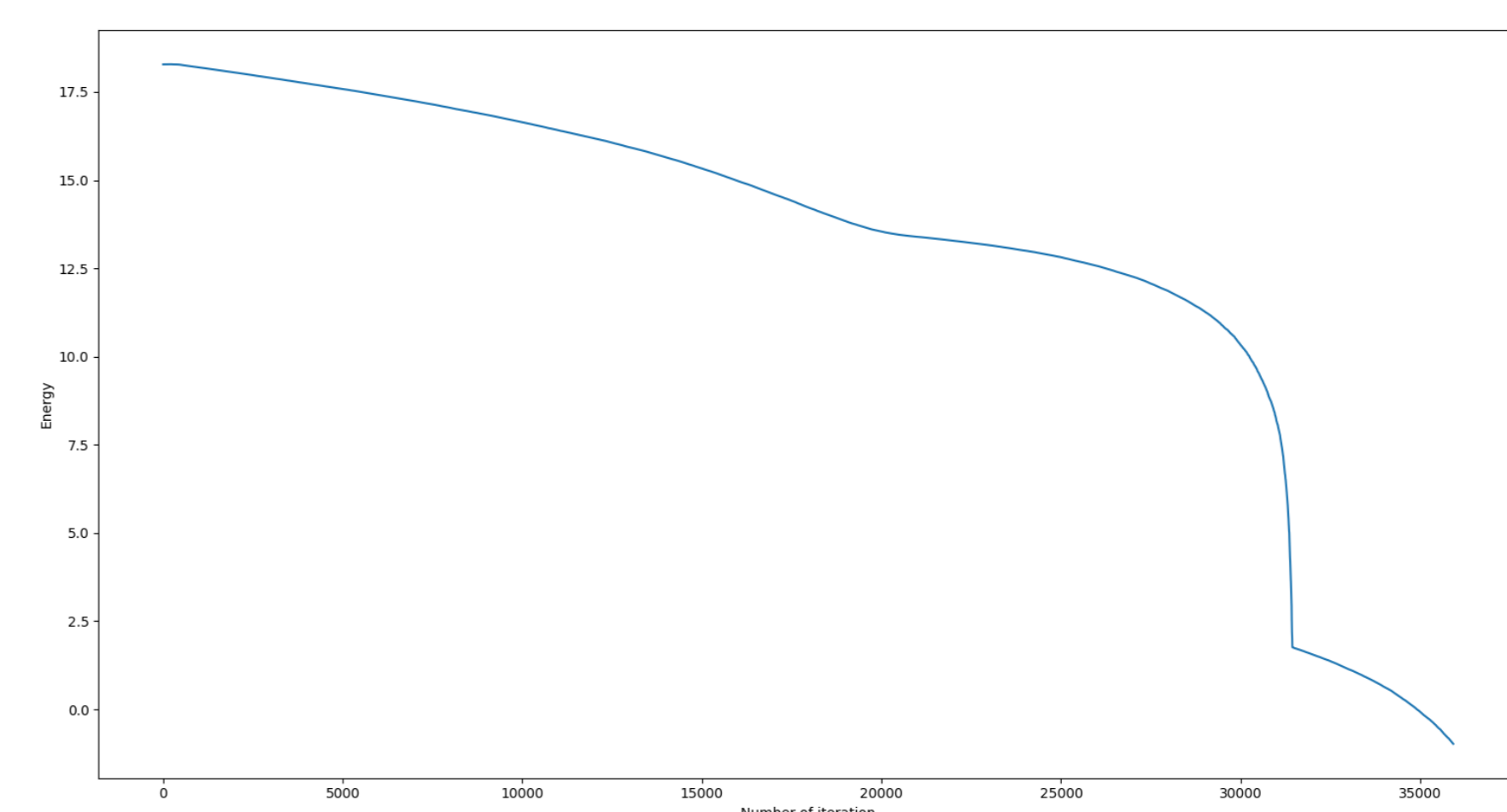


FIGURE – Evolution of the mean square error function

In a second part, we will use a Bayesian approach.

## The inverse problem, bayesian inference

The bayesian approach returns the whole distribution.

$$\mathbb{P}(\mathbf{u}|\mathbf{y}_{obs}) \propto \mathbb{P}(\mathbf{y}_{obs}|\mathbf{u}) \mathbb{P}(\mathbf{u})$$

a posteriori distribution      likelihood      a priori distribution

We will use Markov Chain Monte Carlo methods, to obtain the posterior distributions.

## Conclusions

- ▶ Point model is good enough to serve as a forward model in the solution of the inverse problem.
- ▶ The ultimate goal is to apply a bayesian approach to estimate the a posteriori distribution of the nuclear matter parameters.

## References

- [1] Leo Breiman, PROBABILITY AND STOCHASTIC PROCESSES : With a View Toward Applications
- [2] Pázsit & Enqvist, Neutron noise in zero power systems, Nuclear Engineering, Department of Applied Physics, Chalmers University of Technology, Göteborg, Sweden 2008
- [3] J. M. Verbeke & O. Petit, Stochastic Analog Neutron Transport with TRIPOLI-4 and FREYA : Bayesian Uncertainty Quantification for Neutron Multiplicity Counting, NUCLEAR SCIENCE AND ENGINEERING, Vol. 183, 2016
- [4] Hage & Cifarelli, Correlation Analysis with Neutron Count Distributions in Randomly or Signal Triggered Time Intervals for Assay of Special Fissile Material, NUCLEAR SCIENCE AND ENGINEERING, Vol. 112, 1992